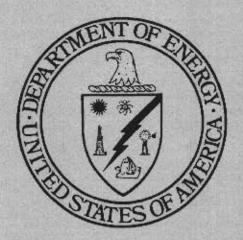


Sandia National Laboratories / New Mexico

PROPOSAL FOR NO FURTHER ACTION
ENVIRONMENTAL RESTORATION PROJECT
SITE 234, STORM DRAIN SYSTEM OUTFALL SITE
OPERABLE UNIT 1309

June 1995

Environmental Restoration Project



United States Department of Energy Albuquerque Operations Office

PROPOSAL FOR NO FURTHER ACTION

Site 234, Storm Drain System Outfall Site Operable Unit 1309

SANDIA NATIONAL LABORATORIES/NEW MEXICO

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1. Introduction

1.1 ER Site Identification Number and Name

Sandia National Laboratories/New Mexico (SNL/NM) is proposing a risk-based no further action (NFA) decision for Environmental Restoration (ER) Site 234, Storm Drain System Outfall Site, Operable Unit (OU) 1309. ER Site 234 is listed in the Hazardous and Solid Waste Amendment (HSWA) Module IV (EPA August 1993) of the SNL/NM Resource Conservation and Recovery Act (RCRA) Hazardous Waste Management Facility Permit (NM5890110518) (EPA August 1992).

1.2 SNL/NM Risk-Based NFA Process

This proposal for a determination of an NFA decision has been prepared using the criteria presented in Section 4.5.3 of the SNL/NM Program Implementation Plan (PIP) (SNL/NM February 1994). Specifically, this proposal will "contain information demonstrating that this SWMU has never contained constituents of concern that may pose a threat to human health or the environment" [as proposed in the Code of Federal Regulations (CFR), Section 40 Part 264.51(a) (2)] (EPA July 1990). The HSWA Module IV contains the same requirements for an NFA demonstration:

Based on the results of the RFI [RCRA Facility Investigation] and other relevant information, the Permittee may submit an application to the Administrative Authority for a Class III permit modification under 40 CFR 270.42(c) to terminate the RFI/CMS [corrective measures study] process for a specific unit. This permit modification application must contain information demonstrating that there are no releases of hazardous waste including hazardous constituents from a particular SWMU at the facility that pose threats to human health and/or the environment, as well as additional information required in 40 CFR 270.42(c) (EPA August 1993).

For a risk-based proposal, an SWMU is eligible for an NFA determination if the NFA criterion established by the SNL/NM permit is met. This criterion, found in Section M.1 of the permit, is as follows: "[T]here are no releases of hazardous waste including hazardous constituents...that pose threats to human health and/or the environment..." This risk-base proposal contains information needed to make the NFA determination.

This proposal is using the technical approach which is the foundation for the SNL/NM corrective action process. The details of the SNL/NM technical approach are provided in Appendix C of the PIP. The first step in the technical approach is the data qualitative review step (the same step used to determine whether the SWMU is eligible for administrative NFA). Should significant uncertainties remain, the assessment of the SWMU continues within the SNL/NM technical approach.

At this site, sufficient data were not available to compare to established action levels or develop site-specific action levels. Background soil samples were collected and analyzed to

develop upper tolerance limits (UTLs) for metals. Site-specific data were collected to compare to existing soil action levels (proposed Subpart S action levels) and UTLs. If site-specific concentrations exceeded the proposed Subpart S action levels or UTLs, then a risk assessment was performed. The site-specific concentrations were compared to the derived risk assessment action levels. Concentrations less than these action levels, either proposed Subpart S action levels, UTLs, or derived risk-based values, triggered this NFA proposal for Site 234.

1.3 Local Setting

SNL/NM occupies 2,829 acres of land owned by the Department of Energy (DOE), with an additional 14,920 acres of land provided by land-use permits with Kirtland Air Force Base (KAFB), the United States Forest Service, the State of New Mexico, and the Isleta Indian Reservation. SNL/NM has been involved in nuclear weapons research, component development, assembly, testing, and other nuclear activities since 1945.

ER Site 234 (Figure 1) is located on land owned by DOE. The outfall is located along the northern embankment of Tijeras Arroyo southeast of Building 981I (Inflatable Building) and a lagoon impoundment in Technical Area (TA) IV.

Surficial deposits in the SNL/KAFB area lie within four geomorphic provinces which in turn contain nine geomorphic subprovinces. Site 234 lies within the Tijeras Arroyo subprovince. The Tijeras Arroyo subprovince is characterized by broad, west-sloping alluvial surfaces and the 50-meter-deep Tijeras Arroyo. The Tijeras Arroyo subprovince contains deposits derived from many sources, including granitic and sedimentary rocks of the Sandia Mountains, sedimentary and metamorphic rocks of the Manzanita Mountains, and sediments of the Upper Santa Fe Group.

2. History of the SWMU

2.1 Sources of Supporting Information

In support of the request for a risk-based with confirmatory sampling NFA decision for ER Site 234, a background study was conducted to collect available and relevant site information. Interviews were conducted with SNL/NM staff and contractors familiar with site operational history.

The following information sources were available for the use in the evaluation of ER Site 234:

- Confirmatory sampling program conducted in September 1994
- Risk analysis for two radionuclides
- One surface radiation survey
- One unexploded ordnance/high explosives (UXO/HE) survey
- Interviews and personnel correspondence
- Historical aerial photographs spanning 40 years
- Personal breathing zone air sampling

2.2 Previous Audits, Inspections, and Findings

In November 1993, the Sandia ER staff recognized Site 234 as an SWMU. ER Site 234 was not listed as a potential release site based on the Comprehensive Environmental Assessment and Response Program (CEARP) interviews in 1985 (DOE September 1987). In addition, Site 234 was not included in the Environmental Protection Agency (EPA) RCRA Facility Assessment (RFA) in 1987 (EPA April 1987) and Site 234 was not included in the Hazard Ranking System (DOE September 1987).

2.3 Historical Operations

The outfall discharged industrial effluent and storm water from TA-IV (Figure 1). Currently, the outfall discharges only storm water. The specific constituents in the industrial effluent are not known. The possible discharge contaminants include chromates, antifoulants, chromium, sodium hydroxide, hydrochloric acid, chromosulfuric acid, diesel, and other petroleum products. Mineral oil is also considered a potential soil contaminant because of a recent release (June 1994) of mineral oil at a similar outfall, Site 232.

3. Evaluation of Relevant Evidence

3.1 Unit Characteristics

The Storm Drain System Outfall is confined to the downstream natural drainage. All releases would be contained in this restricted area.

3.2 Operating Practices

Based on interviews and personnel correspondence, the outfall discharged industrial effluent and storm water from approximately 1978 to 1991. Examination of aerial photographs confirms this time frame but provides no additional information.

3.3 Presence or Absence of Visual Evidence

The approximately 250-foot long outfall and the cement culvert are the only physical evidence of the outfall system. No discoloration of soils was observed during site reconnaissance and soil sampling activities.

3.4 Results of Previous Sampling/Surveys

In 1994, the site was visually surveyed for surface indications of unexploded ordnance and UXO/HE. No UXO/HE were found (SNL/NM 1994a). Also in 1994, a surface radiation survey was conducted on the entire site using an Eberline ESP-2 portable scaler, with an Eberline SPA-8 (2 inch X 2 inch sodium iodide) detector. A 30-second integrated count was performed at each proposed sample location, while scanning the detector over an area

approximately 2 feet in radius around the sample location. The alarm was set at 1.3 times the background count rate. No alarms occurred during the survey. No surface anomalies were detected (SNL/NM 1994b).

3.5 Assessment of Gaps in Information

No environmental sampling data existed for Site 234. If contamination was present, potential constituents of concern (metals, radioactive constituents, and organic constituents), would be expected at shallow depths. Metals and radioactive constituents generally adsorb on soil and precipitate rather than remaining soluble. If organic constituents were introduced in the drainage, they should be detectable in surface or shallow subsurface soils.

3.6 Confirmatory Sampling

A surface (0-6 inches deep) and shallow subsurface (6-36 inches deep) soil sampling program was developed and implemented in September 1994. The Confirmatory Sampling and Analysis Plan (SAP) can be found in Appendix A. Those soil sample results exceeding an action level are summarized in Table 1. A complete list of "hits" or detections and quality assurance (QA) results can be found in Appendix B.

For health and safety purposes, a photo-ionization detector, OVM, was used throughout the field program. The OVM measured no anomalous vapor concentrations.

Surface and shallow subsurface soil samples were collected at the most likely locations of contamination. The inlets to this site are uncontrolled. Two samples were collected at each of four inlets and four samples were collected at the furthest extent of visible erosion and scour (Figure 1). Every sample was analyzed for metals¹, chromium¹⁶, and total petroleum hydrocarbons (TPH). The six subsurface samples also were analyzed for volatile organic compounds (VOCs). Six samples were analyzed for semivolatile organic compounds (SVOCs). As a general check for radioactive constituents, two samples were analyzed for tritium, one sample was analyzed for isotopic uranium and plutonium, and four samples were screened with in-house gamma spectroscopy.

3.6.1 Background Samples for Metals and Radioactive Constituent

UTLs for background metals were calculated from analyses of 24 samples collected in the vicinity of the 11 sites discussed in the SAP (Appendix A). UTLs or background 95th percentiles for background radionuclides were calculated from samples collected throughout KAFB (IT 1994). A discussion of background calculations and supporting data and analyses are included in Appendices C and D.

¹ Although the targe analyte list (TAL) metal analytes include calcium, magnesium, potassium, and sodium, these nontoxic, major cations are not included in the evaluation. They do not pose a significant environmental or human health risk regardless of concentration.

3.6.2 Organic Compounds

No analyses yielded positive detections of organic compounds. All detections were qualified with a "J" (see Table 1), meaning detected below the reportable limit and most detections also were qualified with a "B," meaning detected in the associated blank. None of these qualified detections indicate significant contamination. No TPH was detected.

3.6.3 Metals

Personal breathing zone air sampling was performed to monitor airborne particulate contamination for metals at Site 234. No airborne metal contamination was detected. The maximum local background value for beryllium was 0.53 milligrams per kilogram (mg/kg). Beryllium was not detected above 0.53 mg/kg at Site 234. Mercury, selenium, silver, and chromium⁺⁶ were not detected in any site samples. No other metal samples had concentrations above the local background UTLs. Based on the soil sample data, metals pose an insignificant human health and environmental risk at Site 234.

3.6.4 Radionuclides

Thallium was not detected at Site 234. Plutonium-239/240, plutonium-238, and uranium-235/236 were not detected above the minimum detectable activity (MDA). Uranium-238 and uranium-234 were detected in Sample 234-01-A at 0.44 and 0.50 picocuries per gram (pCi/g), respectively; both were below the base-wide background 95th percentile of 1.1 and 1.0 pCi/g and below the maximum local background values of 0.84 and 0.97 pCi/g, respectively. Radium-226 was detected in Sample 234-01-A at 2.27 pCi/g compared to a base-wide background UTL of 1.94 pCi/g. Additional off-site radiological analyses for radium-226 indicated lower activities than 2.27 pCi/g. Tritium was detected in Samples 234-01-A and 234-05-A at 0.23 and 0.038 pCi/g, respectively.

3.6.5 Quality Assurance Results

As discussed in the Confirmatory Sampling and Analysis Plan (Appendix A), quality assurance samples, including field duplicates, trip blanks and rinsates, were collected as part of the 11-site sampling program. Analyses indicate that the field soil duplicates were comparable to the original soil sample results. The trip blanks and rinsates indicated no significant sampling contamination. QA results can be found in Appendix B. Level I and Level II data verification was conducted on all data, as described in the PIP (SNL/NM 1994).

3.7 Risk Analysis

To further evaluate the site data for radionuclides with activities above background UTLs (or 95th percentiles) or those without background UTLs, risk was analyzed for the combination of tritium and radium-226, assuming the maximum detected activities.

The risk calculations were designed to produce conservatively large estimates of radioactive dose to counter uncertainties in the soil data. This approach facilitates the following decision regarding future activities at Site 234:

- If the conservative estimates based on the soil data result in an unacceptable dose (greater than 10 mrem/year), further investigation and/or remediation will be needed; or
- If the dose estimates are acceptable, the potential for health hazards at the site is extremely low, and further actions will not be needed.

Radionuclide doses were computed using methods and equations promulgated in proposed RCRA Subpart S documentation (EPA 1990). Accordingly, all calculations were based on the assumption that receptor doses from radionuclides result from ingestion of contaminated soil.

Calculation of radionuclide doses required values of dose conversion factors, which are used to convert radionuclide intakes (in units of pCi/year) into effective dose equivalents (in units of mrem/year). Published values of dose conversion factors (Gilbert et al., 1989) exist for tritium and radium-226.

To assure that the computed doses were conservatively large, only the maximum observed activity of each constituent at a site was employed. To consider combined effects, a radiological dose was calculated as the sum of the individual doses.

Following proposed Subpart S methodology, the equation and parameter values used to calculate the summed radioactive dose were:

DOSE =
$$\sum_{i}$$
 [DSR(i) x S(i)]

(1)

where:

DOSE	=	total effective dose equivalent (mrem/yr);
DSR(I)	=	dose-to-soil concentration ratio for the ith radionuclide
		(mrem/yr)/(pCi/g), = I X DCF(I);
S(I)	=	soil concentration of the i th radionuclide (pCi/g);
I	=	soil ingestion rate = 0.2 g/day = 73 g/yr; and
DCF(I)	=	dose conversion factor for the ith radionuclide (mrem/pCi).

The PIP stipulates that, for the purpose of computing media action levels, the total radioactive dose at a site should not be greater than 10 mrem/year (SNL/NM 1994), which corresponds to a cancer risk of less that 10⁻⁶ excess deaths.

The input and results of the risk calculations are presented in Table 2. The summed radioactive dose is less than 10 mrem/year. Therefore, the site is considered to be risk-free in terms of radionuclide contamination.

3.8 Rationale for Pursuing a Risk-based NFA Decision

Surface soil and shallow subsurface soil samples were collected at the uncontrolled inlets of the outfall and at the furthest extent of visible erosion/scour where the discharged effluent would have most likely settled. These areas are the most likely areas for contamination. SNL/NM is proposing a risk-based NFA because representative soil samples from ER Site 234 have concentrations less than action levels; either proposed Subpart S action levels, background UTLs, background 95th percentiles, or derived risk-based values.

In addition

- A site visit in 1993 by ER personnel confirmed the presence of a confined natural drainage with no discoloration in the soils.
- In June 1994, a UXO/HE visual survey was conducted by KAFB Explosives Ordnance Division (EOD) and found no UXO/HE ordnance debris at Site 234 (SNL/NM 1994a).
- In September, 1994, Personal Breathing zone air sampling was performed to monitor airborne particulate contamination for metals at Site 234. No airborne contamination was detected.
- In September, 1994, as part of the surface soil sampling effort at Site 234, a surface radiation survey was conducted (SNL/NM 1994b). No surface anomalies were detected at Site 234.

4. Conclusion

Based upon the evidence cited above, ER Site 234 has no releases of hazardous waste or hazardous constituents that pose a threat to human health and/or the environment. Therefore, ER Site 234 is recommended for an NFA determination.

5. References

5.1 ER Site References

Gilbert, T.L., C. Yu, Y.C. Yuan, A.J. Zielen, M.J. Jusko, and A. Wallo, 1989. Implementing Residual Radioactive Material Guidelines, A Supplement to U.S. Department of Energy Guidelines for Residual Radioactive Material at Formerly Utilized Sites Remedial Action Program and Surplus Facilities Management Program Sites, prepared by Argonne National Laboratory for U.S. Department of Energy, ANL/ES-160, DOE/CH/8901, 203 pp.

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Sandia National Laboratories/New Mexico (SNL/NM), 1994a. "Unexploded Ordnance/High explosives (UXO/HE) Visual Survey of ER Sites Final Report, Albuquerque, New Mexico," Sandia National Laboratories, Albuquerque, New Mexico.

Sandia National Laboratories/New Mexico (SNL/NM), 1994b. "Summary of Radiological Survey Results by SNL Dept. 7714 for ER Sites 50, 227, 229, 230-234, Sandia National Laboratories, Albuquerque, New Mexico," Sandia National Laboratories, Albuquerque, New Mexico.

- U.S. Environmental Protection Agency (EPA), July 1990. "Corrective Action for Solid Waste Management Units (SWMU) at Hazardous Waste Management Facilities, Proposed Rule," *Federal Register*, Vol. 55, Title 40, Parts 264, 265, 270, and 271.
- U.S. Environmental Protection Agency (EPA), 1989. "Statistical Analysis of Ground-Water Monitoring Data at RCRA Facilities, Addendum to Interim Final Guidance," U.S. Environmental Protection Agency, Office of Solid Waste, Waste Management Division, Washington DC.
- U.S. Environmental Protection Agency (EPA), 1989. "Risk Assessment Guidance for Superfund: Volume I, Human Health Evaluation Manual (Part A)." Office of Emergency and Remedial Response, Washington, DC. 20460.

5.2 Reference Documents

Department of Energy (DOE), September 1987. "Comprehensive Environmental Assessment and Response Program, Phase I Installation Assessment Sandia National Laboratories - Albuquerque," Department of Energy Albuquerque Operations Office, Environmental Safety and Health Division, Environmental Program Branch, September 1987.

U.S. Environmental Protection Agency (EPA), April 1987. "Final RCRA Facility Assessment Report of Solid Waste Management Units at Sandia National Laboratories, Albuquerque, New Mexico," Contract No. 68-01-7038, EPA Region VI.

Sandia National Laboratories/New Mexico (SNL/NM), August 1994. "Environmental Restoration Project Information Sheet for Site 229, Storm Drain System Outfall," Sandia National Laboratories, Albuquerque, New Mexico.

U.S. Environmental Protection Agency (EPA), April 1987. "Final RCRA Facility Assessment Report of Solid Waste Management Units at Sandia National Laboratories, Albuquerque, New Mexico," Contract No. 68-01-7038, EPA Region VI.

Sandia National Laboratories/New Mexico (SNL/NM), February 1994. Draft "Program Implementation Plan for Albuquerque Potential Release Sites," Sandia National Laboratories, Albuquerque, New Mexico.

U.S. Environmental Protection Agency (EPA), August 1993. "Module IV of RCRA Permit No. NM 5890110518, EPA Region VI," issued to Sandia National Laboratories, Albuquerque, New Mexico.

U.S. Environmental Protection Agency (EPA), August 1992. "Hazardous Waste Management Facility Permit No. NM5890110518, EPA Region VI," issued to Sandia National Laboratories, Albuquerque, New Mexico.

5.3 Aerial Photographs

Ebert & Associates, Inc., November 1994. "Photo-Interpretation and Digital Mapping of ER Sites 7,16,45,228 from Sequential Historical Aerial Photographs."

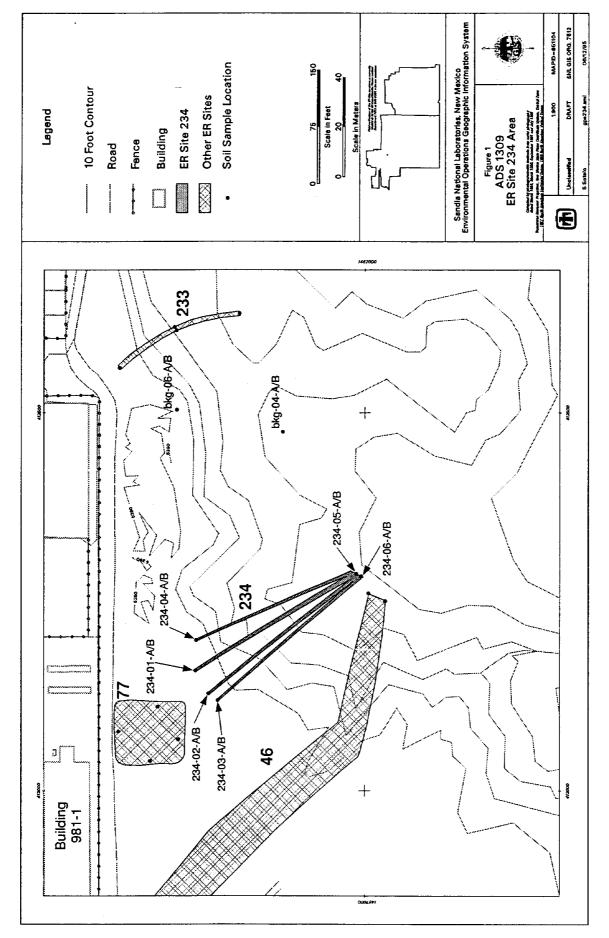


Figure 1. Storm Drain System Outfall Site 234.

Table 1. Site 234 - Results of Shallow Soil Sampling and Analysis

Sample Identifier	Analytical Method	Constituent	Concentration (mg/kg)	Qualifier(s)	Background (mg/kg)	Action Level (mg/kg)
234-01-B	VOCs (8240)	2-butanone	0.002	JB		
234-02-В	VOCs (8240)	2-butanone	0.003	JB	1	
234-03-В	VOCs (8240)	2-butanone	0.005	JB		· · · · · · · · · · · · · · · · · · ·
234-04-В	VOCs (8240)	2-butanone	0.004	JB		
234-05-В	VOCs (8240)	2-butanone	0.003	JB		
234-06-В	VOCs (8240)	2-butanone	0.004	JB		
234-05-A	SVOCs (8270)	Benzo(b) fluoranthene	0.043	J		
234-05-A	SVOCs (8270)	Benzo(a) pyrene	0.048	1		
234-03-A	SVOCs (8270)	Bis (2-ethylhexyl) phthalate	0.28	JВ		
234-05-A	SVOCs (8270)	Chrysene	0.062	1	· · · · · · · · · · · · · · · · · · ·	
234-05-A	SVOCs (8270)	Pyrene	0.034	J		
234-01-A	Tritium (600 906.0)	Tritium	0.23 (pCi/g)			12.6 pCi/g
234-05-A	Tritium (600 906.0)	Tritium	0.038 (pCi/g)			12.6 pCi/g
234-01-A	Gamma Spec (In-house)	Radium-226	2.27 pCi/g		1.94 pCi/g	125 pCi/g

Notes

A "J" qualifier means detected at a concentration below the laboratory reporting limit.

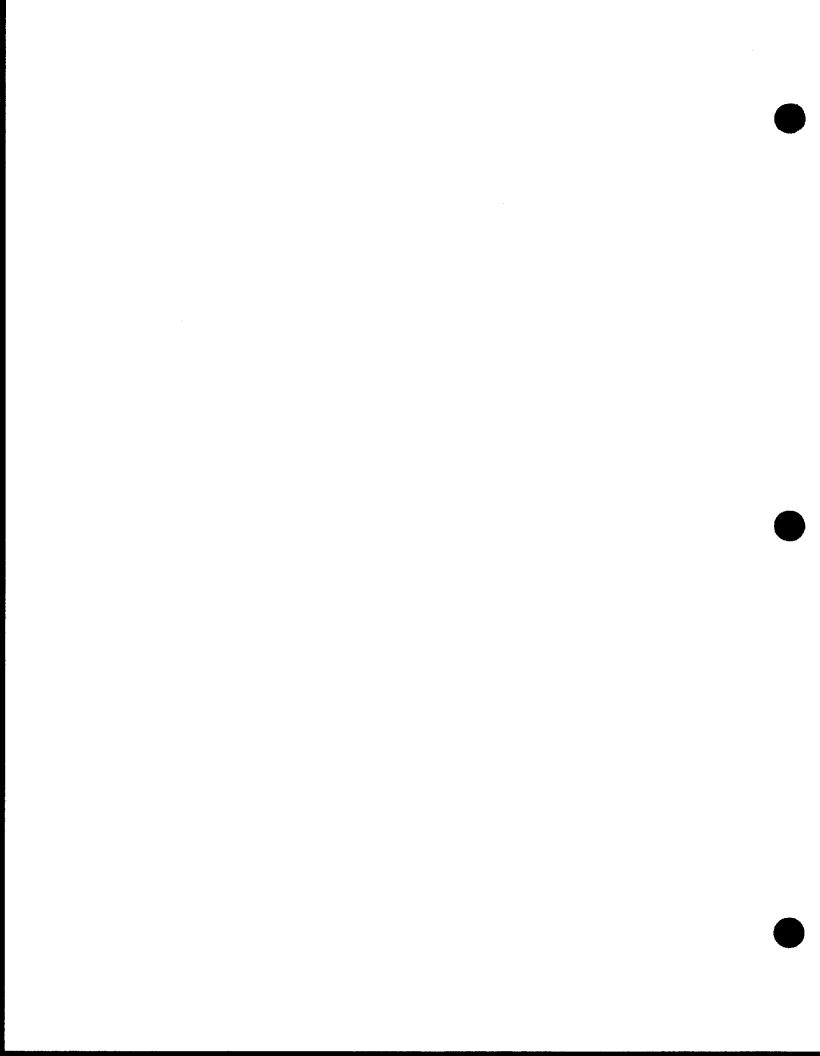
A "B" qualifier means detected in the associated blank sample.

For radium-226, background is the 95 percent upper tolerance level for the base-wide data.

The action levels for tritium and radium-226 are calculated risk-based levels.

Table 2. Risk Calculations for Site 234

Constituent	Activity (pCi/g)	DCF(I) (mrem/pCi)	Individual Dose (mrem/year)	Source of DCF
Radium-226	2.27E+00	1.10E-03	1.82E-01	Gilbert et al., 1989
Tritium	2.30E-01	6.30E-08	1.06E-06	Gilbert et al., 1989
Summed Dose		<u></u>	1.82E-01	



APPENDIX A

Confirmatory Sampling and Analysis Plan

APPENDIX B

Analytical Results

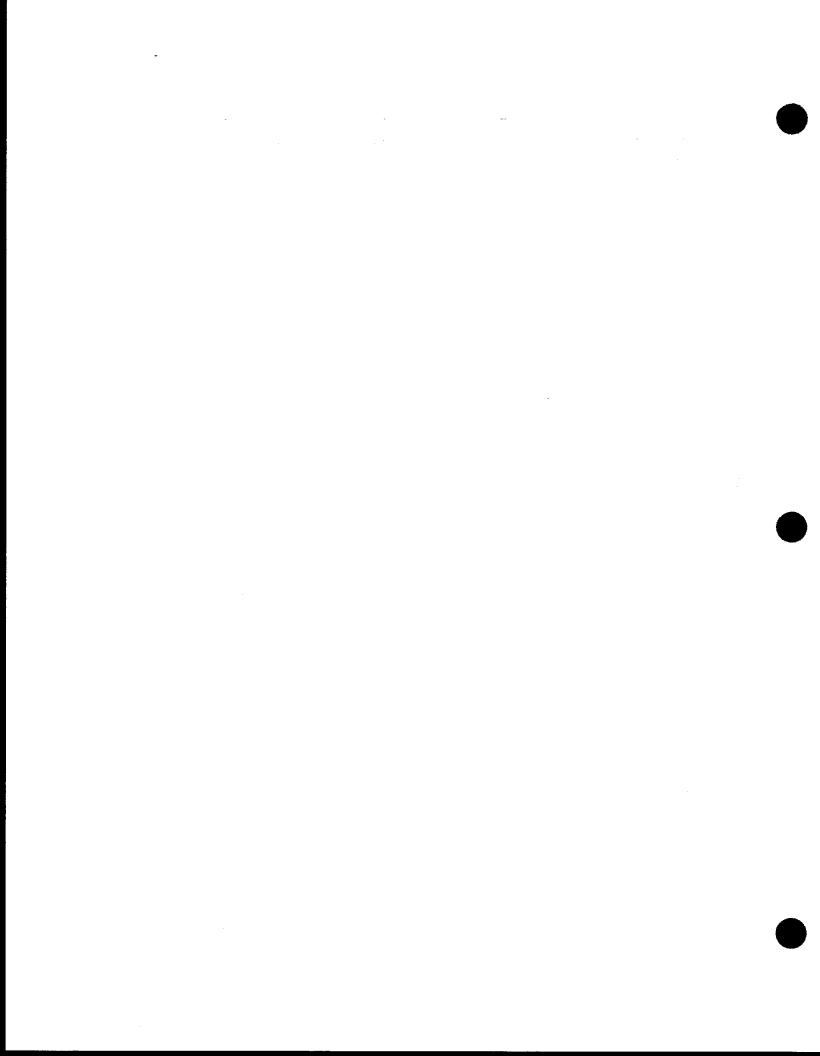
APPENDIX C

Background Calculations for Metals and Radionuclides

APPENDIX D

Probability Plots, Local Background UTL Calculations, and Base-wide Background UTLs for Radionuclides

SAMPLING AND ANALYSIS PLAN FOR ELEVEN SITES IN TIJERAS ARROYO OPERABLE UNIT SANDIA NATIONAL LABORATORIES/ NEW MEXICO



Sampling and Analysis Plan for Eleven Sites in Tijeras Arroyo Operable Unit

Introduction

The purpose of the sampling and analysis described in this plan is to determine the appropriate way to proceed toward closure of 11 (of the 17) sites in the Tijeras Arroyo Operable Unit. Based on the surface and shallow subsurface soil samples and analyses for the constituents of concern (COCs), one of three approaches will be pursued for each site:

- 1. A petition for "No Further Action" (NFA) will be produced for regulatory consideration;
- 2. A voluntary corrective measure (VCM) will be designed and implemented, hopefully followed by an NFA petition; or
- 3. The site assessment and eventual closure will follow the standard RFI/CMS path

Most of the sites covered by this Sampling and Analysis Plan (SAP) are outfalls from the storm water and sanitary sewer systems emanating from Sandia Technical Areas (TAs) I, II, and IV. The general sampling program for the outfalls will be to collect four samples at the head of the outfall, two samples of surface soil (0 to 6 inches deep) and two samples of shallow subsurface soil (18 to 36 inches deep) and four samples (two surface soil and two shallow subsurface soil) at the furthest extent of channel erosion and scour. The analytes for most of the samples are volatile organic compounds, semi-volatile organic compounds (BNAs), metals, chromium +6, for samples where chromium is found in a metals analysis, total petroleum hydrocarbon (TPH), explosives, Total Kjeldahl Nitrogen (TKN), nitrate/nitrite, and Gamma Spectroscopy for radionuclides, isotopic uranium, isotopic plutonium, tritium, and chlorodiphenyls (PCBs).

Sampling Procedures and Volumes

Surface soil samples will be collected with a stainless steel scoopula or trowel and placed in a stainless steel bowl. After at least 1000 ml¹ of soil has been collected, the soil will be thoroughly mixed in the bowl and transferred to two or three 500-ml sample bottles with a stainless steel scoopula. Sample bottles will be labeled accordingly and the appropriate sample information (sample depth, collection date and time, etc.) will be documented on the chain-of custody (COC) after each sample is collected. Samples will then be packaged and cooled to 4 degrees Celsius.

Shallow subsurface soil samples (18-36 inches) will be collected with a 2-inch (minimum) hand auger. A soil sample is collected by turning the auger clockwise and advancing it into the ground until the bucket at the end of the auger (last 6-8 inches) is full of soil or refusal occurs. Several runs with the auger is anticipated in order to obtain the appropriate volume. A hand shovel may also be used to bypass large rocks in order to continue with the auger. The auger is then extruded counter-clockwise from the ground and the soil is removed from the auger and placed in a stainless steel bowl. After 1,125² ml of soil has been collected, the soil will be mixed in the bowl and transferred to two or three 500-ml sample bottles and one 125-ml sample bottle with a stainless steel scoopula. Sample bottles will be labeled accordingly and the appropriate sample information will be documented on the COC after each sample is collected. Samples will then be packaged and cooled to 4 degrees Celsius.

Waste Generation and Equipment Decontamination

Decontamination of sampling equipment will be done between each sample.

Decontamination will include thoroughly washing the inside and outside of the sampling equipment with a spray of ALCONOX™ or LIQUINOX™ and water; rinsing with distilled,

¹The sample volume varies between 1,000 and 1,500 ml depending on the analyses for the sample.

²The sample volume varies between 1,125 and 1,625 ml depending on the analyses for the sample.

Sampling and Analysis Plan for Eleven Sites in Tijeras Arroyo Operable Unit

deionized water; and drying before reusing. No soil waste will be generated. The soil removed from the hand-auger holes, while collecting samples at a depth of 18 to 36 inches, will be return to the hole. The sampling tools, which are scoopulas/trowels, hand-augers, and shovels, will be decontaminated with water and ALCONOX™ after each use. The decon leachate will be stored in capped 1-gallon containers. One or two containers will be used for each site and two to four containers will be used for the background samples. The containers will be labeled as "IDW" and the site number identified on each container. All the containers will be stored at Site 232, a central location. The leachate waste will be disposed according to the analytical results of the soil samples collected at the site.

Site Descriptions

The sites that will be sampled are

- Site 46, Old Acid Waste Line Outfall;
- Site 50, Old Centrifuge Site;
- Site 77, Oil Surface Impoundment;
- Site 227, Bldg. 904 outfall;
- Site 229, Storm Drain System Outfall;
- Site 230, Storm Drain System Outfall;
- Site 231, Storm Drain System Outfall;
- Site 232, Storm Drain System Outfall;
- Site 233, Storm Drain System Outfall;
- Site 234, Storm Drain System Outfall; and
- Site 235, Storm Drain System Outfall.

The site locations are shown in Figure 1. A description of the site history, conditions, previous investigations, and sampling plans are described in the following sections.

Site 46: Acid Waste Line Outfall

The Old Acid Waste Line carried wastes from several buildings in TA I. The waste line begins as a north-south trending, 750-feet long open trench in a grassy field northwest of Building 981-1 in TA IV. No pipe opening is visible at the "head" of the trench. As the trench crosses the field, it turns to the southeast and continues to a non-engineered spillway at the edge of Tijeras Arroyo. The spillway lies on a bank (40 to 50 feet of relief) composed of compacted alluvial sediment. Historical aerial photographs show vegetation, presumably supported by the discharge, growing southeast of the spillway to the active arroyo channel (about 200 feet distance from the spillway). The site is not restricted and is easily accessible.

During use, discharged effluent averaged an estimated 130,000 gallons per day. Use of the line has been discontinued. The line received wastes from plating, etching, and photo processing operations, and cooling tower "blow down". Acids and metals are target contaminants. Chromic acid and ferric chloride are mentioned specifically in the site history, and ferric chloride was found in the soils during a limited sampling event. Various radionuclides, possibly including tritium, uranium, and plutonium were used in TA I.

Building 863 was a source of discharge to the Acid Line. The information sheet for ER Site 98 (Building 863, TCA Photochemical Release: Silver Catch Boxes) indicates the presence of trichloromethane, silver, and photo-processing chemicals with an ammonia-like odor. The waste solution from the silver recovery unit reportedly was discharged to the Old Acid Waste Line, which is the only specific information about chemical discharges.

The site has been visually surveyed for surface indications of unexploded ordnance and high explosives (UXO/HE). No UXO/HE were found. Also, a surface radiation survey was

Sampling and Analysis Plan for Eleven Sites in Tijeras Arroyo Operable Unit

conducted on the entire site. No surface radiation anomalies were detected.

The sampling program includes four samples collected at the "head" of the site outfall (by the fire extinguisher training area west of TA IV) and four samples collected by the spillway into the Tijeras Arroyo drainage (Figure 1). Every sample will be analyzed for tritium, metals, chromium +6 (if chromium is detected), TKN, and nitrate/nitrite. Half the samples will also be analyzed for semi-volatiles and cyanide. Additionally, all the subsurface samples will be analyzed for volatiles. The analytes are listed in Table 1. A "4" on the table indicates that ALL the samples will be analyzed

for that specific analyte whereas a "2" on the table indicates half the samples will have additional analyses for the analyte listed.

Site 50: Old Centrifuge

Site 50, Old Centrifuge, was an outdoor, rocket propelled centrifuge that was used in the early 1950s to test units under G forces. The facility is located east of the TA II fence in a slight depression on top the escarpment northwest of Tijeras Arroyo. The concrete centrifuge pad has a diameter of 80 to 90 feet. The site has a 7-foot high wooden retaining wall on the north, east, and south sides. The west side is open. The centrifuge arm assembly, which has a 20-foot radius, is sitting outside the wall to the north and appears to be intact. Control wiring to the center axis of the centrifuge was suspended from a cable between two telephone poles on the north and south side of the pad. The control wiring went to a bunker located to the southwest over the escarpment. The bunker had a electrical transformer containing PCB. The electrical transformer has been removed. The pad was not stained and no spills or leaks were reported.

The centrifuge was rocket driven by two T40 6-KS-3000 or two Deacon 3.5DS-5700 solid rocket motors. The combustion byproducts produced by these rocket motors were carbon dioxide, carbon monoxide, water, hydrochloric acid, aluminum oxide, and possibly barium oxide. No other HE is known or suspected at the site. The rocket orientation would expel combustion byproducts towards the retaining wall and the opening to the west. The rocket propellant would be consumed in the rocket motor case. Under normal operating conditions, no unburned propellant would be released.

In 1987, a reconnaissance investigation at five potential contaminated sites, including the Old Centrifuge Site, was conducted by the ER Project. Samples were analyzed for uranium, TNT, HSL inorganics, TCLP constituents, and EP Toxicity constituents. Metals, including barium, were detected at concentrations well below regulatory action levels. Total uranium concentrations were typical of area background levels. TNT, pesticides, PCBs, herbicides, and semi-volatiles TCLP compounds were not detected.

Prior to sampling, the surface will be surveyed for radiation. If contamination exists, it is expected to be around the edge of the centrifuge pad at the surface, probably along the open west side. The constituents of concern are metals (specifically lead, beryllium, and barium), depleted uranium, and high explosives. Four surface samples and four subsurface samples will be collected. The sampling locations will be biased toward the west side of the site because that is the open side (Figure 1). All surface samples will be analyzed for all the COCs. One-half of the subsurface samples will be analyzed for uranium and high explosives. All four subsurface samples will be analyzed for metals.

Site 77: Oil Surface Impoundment

The Oil Surface Impoundment Site is outside the TA IV fence, southeast of Building 981-1. The surface impoundment, which was constructed in the 1970's, is used to catch waste water from accelerators. At the time of the RCRA facilities environmental survey, the impoundment was unlined. Since then the impoundment was drained. Soil samples were analyzed for PCBs and

Sampling and Analysis Plan for Eleven Sites in Tijeras Arroyo Operable Unit

solvents. Based on the analytical results, the impoundment was determined to be clean. Subsequently, the impoundment was lined with geotextile and is now regulated under Sandia's Surface Water Discharge Program.

This site will not require UXO/HE or radiation surface surveys. Minimal confirmation sampling and analysis is proposed to verify that the site is clean. Three surface and three shallow subsurface samples are proposed. The samples will be collected along the perimeter of the existing lined pond (Figure 1). All the samples will be analyzed for PCBs. The subsurface soil samples also will be analyzed for volatile organic compounds (Table 1).

Site 227: Bunker 904 Outfall

Site 227 is an inactive outfall from the septic system for Building 904 (ER Site 48) in TA II. The site starts where the discharge exits the septic tank piping system, approximately 100 feet northeast of the southernmost point of TA II. The extent of the area influenced by the discharge may include the bank of Tijeras Arroyo below the outfall and some area between the outfall and the main channel of Tijeras Arroyo. The site is along the eastern edge of ER Site 45.

Building 904, built in 1948, was used for weapons assembly, HE testing, photo processing, and various other testing. Sanitary wastes were discharged to a septic tank, and other wastes were discharged to the outfall.

Mineral oil is also being considered a potential soil contaminant at all outfalls along the Tijeras Arroyo due to a recent release (June 1994) of mineral oil at Outfall 232 and vague historical records.

Possible soil contaminants are explosives, radioactive materials from weapons processing, including tritium, uranium, and plutonium, solvents (acetone, methylene chloride, methyl ethyl ketone, carbon tetrachloride, toluene, xylene, hexane, alcohols), and inorganics (ammonium hydroxide, barium, cadmium, silver, chromium, titanium, cyanide).

Access to this site is along the TA II perimeter road. This site is within the TA II testing exclusion zone. The best days to sample are generally Friday, Saturday, and Sunday, when testing ceases. Bruce Berry (telephone 845-8018) must be contacted to gain permission and access to this site. Prior to sampling

- tumbleweeds will be cleared from locations to be sampled and placed adjacent to the drainage;
- 2. these locations will be visually scanned for UXO/HE; and
- 3. these locations will be screened for surface radiation anomalies.

The proposed sampling program is to collect four surface soil samples and four shallow subsurface samples. Two surface and two subsurface samples will be collected at the outfall. The other two surface and two subsurface samples will be collected at the furthest visible channel erosion and scour (Figure 1). The analytes are listed in Table 1.

Sites 229 - 235: Storm Drain Systems Outfalls

These sites consist of the discharge areas at seven outfalls along the northern embankment of Tijeras Arroyo. The outfalls discharged industrial effluent and storm water from TAs I, II, and IV. Presently they only discharge storm water. The outfalls receive runoff from Site 96 (Storm Drain System) and other engineered drain systems within the three TAs. The sites are along approximately 3/4 miles of the embankment.

The specific constituents in the industrial effluent at these sites are not known. The possible discharged contaminants include chromates, antifoulants, chromium, sodium hydroxide, hydrochloric acid, chromosulfuric acid, diesel, and other petroleum products. To cover this array of possible contaminants, soil samples will be analyzed for volatiles (subsurface samples only), semi-volatiles, metals and chromium of chromium is found in the metals analysis.

Sampling and Analysis Plan for Eleven Sites in Tijeras Arroyo Operable Unit

Mineral oil is also being considered a potential soil contaminant at all outfalls along the Tijeras Arroyo due to a recent release (June '94) of mineral oil at Outfall 232 and vague historical records. Therefore, soil samples will also be analyzed for TPH.

At Sites 229 through 234, prior to sampling

- tumbleweeds will be cleared from locations to be sampled and placed adjacent to the drainage;
- 2. these locations will be visually scanned for UXO/HE; and
- 3. these locations will be screened for surface radiation anomalies.

Site 229 is due east of the footings of the old guard tower and the south "corner" of the TA II fence. It discharges near the top of the embankment through the center of ER Site 45. Access to this site is along the TA II perimeter road. This site is within the TA II testing exclusion zone. The best days to sample are generally Friday, Saturday, and Sunday, when testing ceases. Bruce Berry (telephone 845-8018) must be contacted to gain permission and access to this site. Because this site discharges from TA II, various radionuclides, possibly including tritium, uranium, and plutonium are of concern. Four surface soil and four subsurface soil samples will be collected at this site (Figure 1). The analytes are listed in Table 1.

Site 230 is west of Building 970 in TAIV. A drain pipe discharges into a bowl-shaped concrete structure adjacent to Building 970A. Flow from this structure is directed to a drain and flume located approximately 120 feet further west. The flume carries the flow to a discharge point slightly above the base of the arroyo embankment. Doug Bloomquist (845-7455) must be contacted to ensure that no laser testing is being performed in the area. Four surface soil and four subsurface soil samples will be collected at this site (Figure 1). The analytes are listed in Table 1.

Site 231 is west of Building 970 in TA IV. A drain pipe discharges to a concrete flume near the top of the embankment. The flume carries the flow to a discharge point near the base of the slope. Doug Bloomquist (845-7455) must be contacted to ensure that no laser testing is being performed in the area..Four surface soil and four subsurface soil samples will be collected at this site (Figure 1). The analytes are listed in Table 1.

Site 232 consists of two outfalls. One outfall is south of Building 970A, east of the lined lagoon. A drain pipe discharges to a concrete flume near the top of the embankment. The flume carries the flow to at discharge point near the bottom of hillside. On June 1, 1994, about 150 to 350 gallons of mineral oil was spilled into this outfall through the storm water drain by building 986. The day after the spill the site was screened for radiation and UXO/HE. No surface radiation anomalies or UXO/HE were found. Also, four surface soil and four subsurface soil samples were collected. The samples were sent to Quintera Laboratory in Denver for analysis for organics, metals, chromium¹⁶, and gamma spec. Other than TPH from the mineral, no contaminants were detected. A Voluntary Corrective Measure was conducted in July and August to remove soil contaminated with mineral oil above 100 mg/kg of TPH.

The second outfall in Site 232 also is south of Building 970A, west of lined lagoon, and approximately 120 feet east of the other Site 232 outfall. Discharge occurs from a concrete structure opening near base of embankment. Access to the site is along the road outside the south side of TA IV. Four surface soil and four subsurface soil samples will be collected at this drainage Figure 1). The analytes are listed in Table 1.

Site 233 is south-southwest of Building 986. Near the top of an escarpment, a small metal drain pipe discharges to an open drain which directs flow within another pipe before discharging near the base of the hillslope. Access to the site is along the road outside the south side of TA IV. Four surface soil and four subsurface soil samples will be collected at this site (Figure 1). The analytes are listed in Table 1.

Site 234 is southeast of Building 981I (Inflatable Building) and a tagoon impoundment (Site 77).

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The site discharges into a steep-sided, deeply incised channel cut into the hillside. The drainage channel splits directly uphill of a tree. Access to the site is along the road outside the south side of TA IV. Both channels will be sampled. Six surface soil and six subsurface soil samples will be collected at this site (Figure 1). The analytes are listed in Table 1.

Site 235 is immediately downstream of a large concrete spillway on the northeast side of Pennsylvania and south of the Skeet Range, at the point where the road comes off the north bank of the arroyo and descends into the channel. The flow moves in a confined channel after dropping down the spillway. The site has been cleared for visible surface UXO/HE and screened for surface radiation with no anomalies detected. This channel is considerably larger than the other outfall sites. Six surface soil and six subsurface soil samples will be collected at this site (Figure 1). The analytes are listed in Table 1.

Background

Background soil concentrations for organic contaminants should be negligible. Background concentrations for total metals and radionuclides must be determined for comparison to concentrations found at the sites. Twelve locations have been identified to collect samples for background determination (Figure 1). At each of these sites, one sample will be collected at a depth of 0-6 inches and a second sample collected at 18-36 inches (Table 1). In addition, the background study report prepared by International Technology Corporation (May 1994) will also be used to evaluate the data.

Quality Assurance

As shown in Table 1, quality assurance samples will include the following:

- Field "duplicates" on more than 10 percent of the samples. These samples will be collected adjacent to the original surface soil sample and in the same hole as the original subsurface soil sample;
- Field soil blanks for more than 10 percent of the VOC analyses. These sample will be obtained from Sample Management Office (SMO) and will contain no VOCs; and
- One rinsate blank. All rinsate will be composited in one container. A sample of the rinsate will be analyzed for all constituents. The disposal method for the rinsate will be determined by the analytical results on this sample.

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* Analyze for Cr* only if Cr is detected in metals analysis

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Appendix B Analytical Results

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	Vickel	8	80	9	വ	1	8	┢	2	8	Η-	0	9		46S-muinest	-	Γ	Τ		Τ	Τ	T	Τ	Τ		Τ	Τ
	Мегсигу	2	QV	9	L		9	Q	Q	QN	QN	AD	Q		362\285.muins1	<0.013											
	Manganese	230	220	140	130	210	180	130	150	260	210	260	150		85S-muina1L	10		T	T		T		T	T	T	T	†
	muisəngaM	4600	4300	2400	2200	3500	3200	3500	4100	4400	3500	4800	2300		86S muinotul	< 0.008											
5	реәд	L	9.4	Ц		12	8.2	8.2			9.1		6.5		042\239\240	<0.004											
Site 234 Soil Results	lron	11000	11000	12000	12000	9500	9000	8800	10000	12000	13000	13000	8800		աս նի 1	0.23								0.038			
34 So	Copper	9.1	ဥ	9.3	9.6	13	9.8	8,5	7.2	9.5	9.8	11	9.6		8SS-muibsA									8.0			
Site 2:	. JisdoD	4.7	4.7	4.1	5.7	4.1	3.5	3.9	4.1	4.8	4.5	4.9	3.6		გავ-muibeჩ						Γ			0.7			
	muimondO	7.4	7.3	6.9	7	-		5		ı	6.7	9.9	5.4		822-muibsA	2.3	NS							SN	SN		
	muiolsO	42000	46000	50000	31000	30000	65000	61000	48000	32000	27000	34000	31000		Çt₁6	2	QN	QN	QN	QN	Q	QN	Q.	QN	QN	Ð	Q
	Cadmium	2	7	7	<u>س</u>	m	7	7	7	7	က	က	2		SniS	64	64	64	77	67	57	22	22	70	64	73	47
	Beryllium		0.4				_	_		0. 4.	0.3	0.5	0.5		muibeneV	23	24	24	24	18	21	24	30	22	25	28	18
	muinsB	210	190	140	9	188	210	240	220	8	180	220	150		muilledT	QN	QN	g	S	Q.	QN	QN	Ω	S	õ	Q	QN
	SinesiA	1.6	1.6	5.3	-	.8	4.8	6.3	5.4	9	0.9	~	-	I	wnipoS	450	480	320	430	8	290	320	340	300	320	360	250
	YnomitnA	16	13	80	8	12	=	=	=	2	F	`Ι	7		Silver	_	_	_	_	_	-		2		S	_	QN
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	eitimabl aldma2	234-01-A	234-01-B	234-02-A	234-02-B	234-03-A	234-03-B	234-04-A	234-04-B	234-05-A	234-05-B	234-06-A	234-02-B		Sample Identifier	234-01-A	234-01-B	234-02-A	234-02-B	234-03-A	234-03-B	234-04-A	234-04-B	234-05-A	234-05-B	234-06-A	234-06-B

Concentrations in mg/kg
Activities in pCi/g
Sample Identifier XX-XX-A · surface soil samples
Sample Identifier XX-XX-B · subsurface soil samples

	Hqī					I		S	81					L									Q N
	səuəl/Xylenes																						0.001
	Styrene																					0.001 J	
	Ругепе	0.040 J						0.19 J	0.28 J														
	Phenanthrene	0.055 J	0.051 J					0.17 J	0.18 J														
	Methylene Chloride																			0.003			
ituents	Finoranthene	0.066 J	0.038 J					0.23 J	0.20														
c Const	Di-n-octyl phthalate													0.16 J						-		-	
Organi	Сһгуѕепе							0.11 J	0.12 J			,											
ults for	Benzo(b)fluoranthene		-					0.16 J	0.16 J										-				
ice Resi	Benzo(a)pyrene							0.050 J	0.092 J														
uality Assurance Results for Organic Constituents	Benzo(a)anthracene							0.071 J	0.006	┢													
Quality	enofec				0.006 J													0.019	0.015	-		0.015	0.010
	-A-Methyl-2-pentanone			0.001 J														0.002					
	S-Hexanone																	0.003 J		Ė			
	2-Butanone			0.007 J	0.006 J	0.004 J	0.005 J			0.006 J	0.006 J	0.006	0.006 J	0.003 JB		0.006 JB	0.004 JB	0.010 B (0.009 JB	0.004 JB	0.007 JB	0.007 JB	0.005 JB
	Sample Type	original	duplicate	trip blank	trip blank	trip blank (trip blank (trip blank (rinsate (
	Sample Identifier	227-01-A	227-01-A	227-01-B	227-01-B	227-04-B	227-04-B	229-01-A	229-01-A	229-02-B	229-02-B	229-03-B	229-03-B	230-04-B		235-02-B	235-02-B	$\overline{}$	Site 229	Site 230	Site 232	⊢	Site 235

	Quality Assurance Results for Inorganic and Radiological Constituents																
Sample Identifier	Sample Type	Aluminum	Antimony	Arsenic	Barium	Beryllium	Cadmium	Chromium	Cobaít	Copper	Iron	Lead	Manganese	Mercury	Nickel	/anadium	Zinc
227-02-A	original	5800	9.3	5.9	180	ND	2.1	6.6	4.1	7.8	13000	7.5	160		5.4	27	51
227-02-A	duplicate			1.4	150	0.25	2.5	6.4	4.1	13	14000	9.1	170	ND	5.9	28	51
227-03-B		5100		0.92	140	ND	2.1	5.9	4.5	11	13000	7.5	200	ND	5.4	25	48
227-03-B				5.6	140	0.25	2.9	7.4	4.6	10	16000	8.9	230	ND	5.9	33	50
229-04-A		8100	13	5.7	150	0.32	2.3	8.0	4.2	7.9	13000	12	210	ND	6.3	24	55
229-04-A			12	1.5	140	0.30	2.2	8.0	4.2	7.7	12000	11	190	ND	6.2	24	52
230-04-B	3	1500	3.3	1.6	130	ND	0.61	2.3	ND	18	3500	4.2	110	ND	3.0	9.1	82
230-04-B			4.9	1.7	140	ND	0.68	3.1	2.5	15	4500	4.1	120	ND	3.4	9.7	71
235-01-A		3600	6.2	5.1	150	ND	2.7	6.0	8.4	6.6	20000	7.6	210	ND	4.5	36	66
235-01-A			5.3	1.3	160	ИD	1.6	4.2	5.7	6.5	12000	9.4	180	ND	4.4	22	66
50-01-B	original	3100	6.5	2.1	110	0.25	1.3	4.1	3.9	6.2	7600	6.6	130	ND	4.5	17	18
50-01-B	duplicate	3900	7.5	2.0	110	0.26	1.3	4.3	4.0	5.7	8800	5.9	150	ND	4.2	18	21
50-02-A	original	5800	12	4.2	220	0.38	1.6	5.2	4.3	12	6700	25	210	ND	7.1	11	69
50-02-A	duplicate	7000	14	6.4	280	0.55	2.2	8.3	6.1	17	9000	35	290	0.04	9.4	18	61
Bkg-05-A		6400	13	5.7	210	0.53	1.8	6.1	6.6	14	10000	16	330	ND	8.9	22	37
Bkg-05-A	duplicate	5900	12	7.6	190	0.50	1.7	6.0	6.3	14	10000	16	320	ND	8.7	24	36
Site 235	rinsate	ND	ND	ND	ND	ND	ND	ND	ND	ND	ND	ND	ND	ND	ND	ND	ND

											Notes on Quality Assurance Data
Sample Identifier	Sample Type	TKN	NO ₃ /NO ₂	Potassium 40	Lead 212	Lead 214	Plutonium 239/240	Uranium 238	Uranium 235/236	Uranium 234	Explosive residues were not detected in Site 50 duplicate sample Hexavalent chromium was not detected in five duplicates and one decon rinsate
227-02-A		400	2.7								Cyanide was not detected in two
227-02-A		320	9.3								duplicates and one decon rinsate
227-03-A							0.004	0.4	0.15	0.61	000
	duplicate							0.67	0.023	0.67	PCBs were not detected in one Site 77
227-03-B	3							0.72	0.11	0.72	duplicate sample
227-03-B		220	ND								Tritium and Plutonium-238 were not
227-03-B				27.8	0.71	0.7					detected in four duplicate samples
227-03-B		190	1.4								dimples
229-01-A							0.007	0.45	0.17	0.67	Selenium, silver, and thallium were not
229-01-A								0.73	0.034	0.6	detected in any quality assurance
229-03-B					Ï			0.45	0.058	0.45	samples
229-03-B	duplicate							0.99	0.06	1	
		·									

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Appendix C Background Calculations for Metals and Radionuclides

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Appendix C. Background Calculations for Metals and Radionuclides

To evaluate metals data, 24 background samples were collected for metals analyses.⁴ Distribution analyses was performed first by constructing histograms. The histograms indicated a parametric distribution. Outliers were screened in a two-step process as described in the base wide background report (IT 1994). The first step is to perform an "a priori" screening for very high values relative to the rest of the data set. This is qualitatively performed by visually examining a column of sorted values. Maximum values that are a factor of 3 or 4 times higher than their nearest neighbor are removed from the data set during this step. None of the anomalous values were deleted by the "a priori" process.

The second step, from EPA, 1989, determines whether an observation that appears extreme fits the data distribution. A statistical parameter, T_n is calculated:

$$T_n = \{X_n - X_a\}/S$$

where:

 X_n = questionable observation;

 X_a = sample arithmetic mean; and

S = sample standard deviation

 T_n is compared to a table of one-sided critical values for the appropriate significance level (upper 5 percent) and sample size from a table provided in EPA 1989. Extreme concentrations for barium, calcium, chromium, copper and nickel were identified as outliers and were excluded from the data set. These anomalous values may have resulted from laboratory or sampling error.

Probability plots were then replotted to determine whether the data fit normal or lognormal populations. These plots are shown in Appendix D. The UTL⁵ was calculated for data sets that fit a normal or lognormal distribution. Data sets are provided in Appendix D. As recommended by EPA, a tolerance coefficient value of 95 percent was used (EPA 1989). Most metals background data fit lognormal distributions. Iron and zinc data fit normal distributions. UTLs were not calculated for mercury, selenium, and silver because mercury and selenium were not detected and silver was detected only once in the 24 background samples. The beryllium background data did not fit a normal or lognormal distribution. The maximum value in a data set is commonly taken as the UTL in a non-parametric setting (Guttman, 1970). The maximum background beryllium concentration was 0.53 mg/kg.

Base-wide background UTLs for radionuclides were established by International Technology (IT) Corporation to compare and evaluate radionuclide data (IT, 1994). A table is provided in Appendix

²These data are referred to as local background data. The data collected throughout Kirtland Air Force Base (KAFB), with most of the data collected within SNL/NM technical areas, are called base-wide background data (IT 1994).

 $^{^{3}}$ UTL = x + K•S, where:

UTL = Upper tolerance limit;

x = Sample arithmetic mean (for normal distribution), sample geometric mean (for lognormal distribution);

S = Sample standard deviation; and

K = One-sided normal tolerance factor (95 percent for these evaluations).

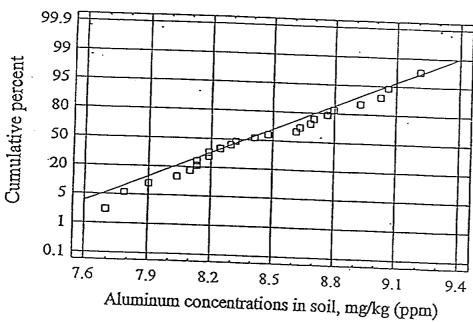
D with radionuclide background data and the corresponding UTLs. The maximum activity from the six local background samples for isotopic plutonium and isotopic uranium was used as an additional method to evaluate the data. Also, in-house gamma spectroscopy was performed on all 24 background samples and indicated low levels of radioactivity but no significant contamination.

Appendix D Probability Plots, Local Background UTL Calculations, and BaseWide Background UTLs for Radionuclides

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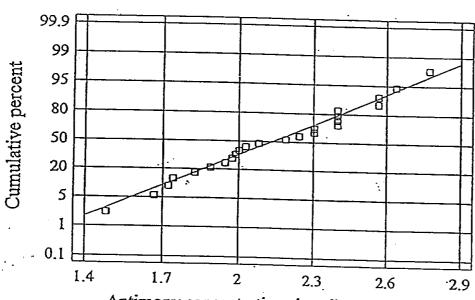
Statistics for log(Aluminum) cage = 8.42942 ian = 0.36529 metric mean = 0.41976 iance = 0.170246 ndard deviation = 0.412609 ndard error = 0.0842235 imum = 7.69621imum - 9.21034 je = 1.51413 or quartile = 8.13153 or quartile = 8.73178 rquartile range = 0.600253 ness = 0.132255 . skewness = 0.26451 osis = -0.792361kurtosis = -0.792361f. of variation = 4.89487 = 202.306

Lognormal Probability Plot for Aluminum



ummacy Statistics for log(Antimony) /erage = 2.14609 :dian = 2.13275 >de = 2.3979 :Ometric mean = 2.12004 riance - 0.113831 andard deviation = 0.337389 andard error = 0.0608692 nimum = 1.4816 ximum = 2.77259 nge = 1.29098 wer quartile = 1.91649 per quartile = 2.3979 terquartile range = 0.481405 ewness = -0.040772 nd. skewness = -0.0815441 rtosis = -0.744171 nd. kurtosis = -0.744171 of variation = 15.7211 n = 51.5062

Lognormal Probability Plot for Antimony



mary Statistics for log(Arsenic)

= 24

Arge = 1.038

Itan = 0.031963

Itan = 0.291153

Itance = 0.291153

Itance = 0.291153

Itance = 0.291153

Itance = 0.405465

Itance = 0.405465

Itance = 0.405465

Itance = 1.41908

Itance = 1.41908

Itance = 0.530628

Itance = 1.41908

Itance = 1.73162

Itance = 1.73162

Itance = 1.73162

Itance = 1.73162

Itance = 0.463036

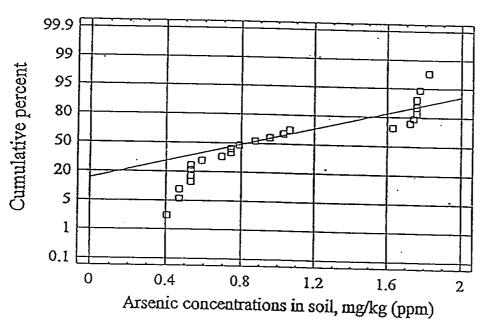
Itance = 0.926071

Itance = 0.926071

Itance = 1.58507

Itan

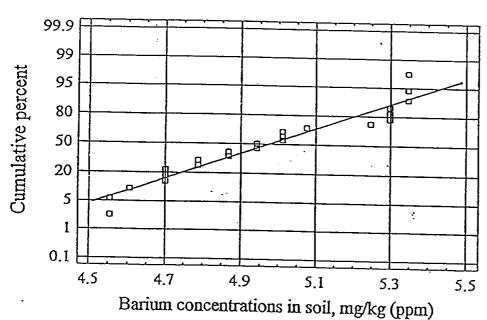
Lognormal Probability Plot for Arsenic



amary Statistics for log(Barium)

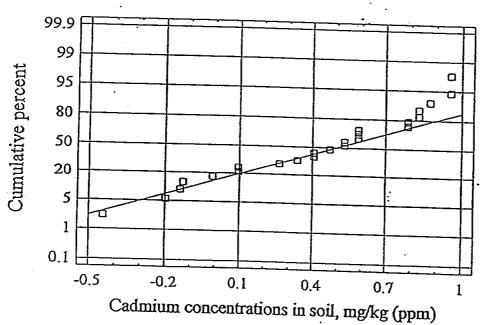
int = 23 !cage = 4.96948 !ian = 4.94164 !e = 5.34711 metric mean = 4.96236 !ance = 0.0740602 ndard deviation = 0.27214 ndard error = 0.0567451 !mum = 4.55388 !mum = 5.34711 ge = 0.793231 er quartile = 4.70048 er quartile = 5.29832 !rquartile range = 0.597837 mess = 0.0653415 !. skewness = 0.127931 !osis = -1.30542 !. kurtosis = -1.27794 !f. of variation = 5.47622 = 114.298

Lognormal Probability Plot for Barium



Summary Statistics for log(Cadmium) ge = 0.416764ledian = 0.500316 lode = eometric mean = ariance = 0.159937 tandard deviation = 0.399922 tandard error = 0.0816337 inimum - -0.446287 aximum = 0.955511 ange = 1.4018 ower quartile = 0.0953102 oper quartile = 0.788457 sterquartile range = 0.693147 cewness = -0.506707:nd. skewness = -1.01341 ictosis = -0.674504 ind. kurtosis = -0.674504 eff. of variation = 95.9587 m = 10.0023

Lognormal Probability Plot for Cadmium

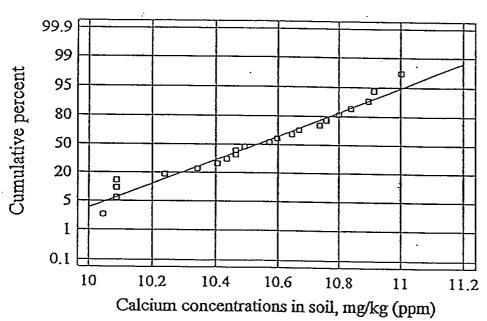


mmary Statistics for log(Calcium) unc = 23 erage = 10.5579 dian = 10.5713de = 10.0058ometric mean = 10.5532 ciance - 0.10513 andard deviation = 0.324237 andard error = 0.0676081 nimum = 10.0432<imum = 11.2645 nge - 1.22121 ver quartile = 10.3417 per quartile = 10.7996 :erquartile range = 0.457833 :wness = 0.109797 id. skewness = 0.214971tosis = -0.415646 id. kurtosis = -0.406895

ff. of variation = 3.07103

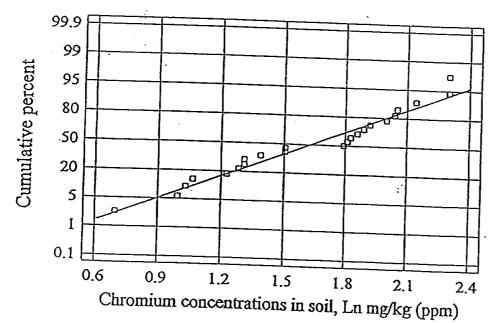
· = 242.832

Lognormal Probability Plot for Calcium



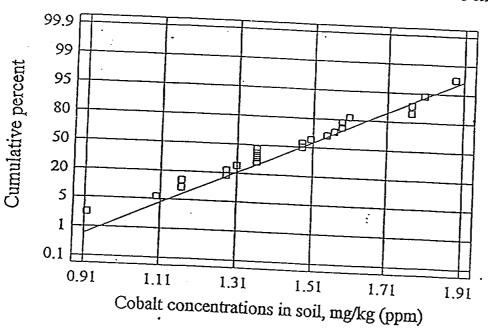
ry Statistics for log(Chromium) erage = 1.61841 dian = 1.79176 ometric mean = 1.55042 riance - 0.204195 andard deviation = 0.451879 andard error = 0.0942233 nimum = 0.693147cimum = 2.30259 1ge = 1.60944 er quartile = 1.28093 er quartile = 2.00148 erquartile range = 0.720546 wness = -0.274151 id. skewness = -0.536757 tosis = -0.905395 d. kurtosis = -0.886332 ff. of variation = 27.9211 = 37.2235

Lognormal Probability Plot for Chromium



Summary Statistics for log(Cobalt) werage = 1.29969 (edian = 1.42129 íode = eometric mean = ariance - 0.574775 tandard deviation = 0.758139 tandard error = 0.154754 inimum - -2.07944 aximum = 1.88707 ange = 3.96651 ower quartile = 1.28093 per quartile = 1.58924 sterquartile range = 0.308301 :ewness = -(.13299 nd. skewness = -8.26598 ictosis = 18.9091 nd. kurtosis = 18.9091 eff. of variation = 58.3324m = 31.1925

Lognormal Probability Plot for Cobalt



mmary Statistics for log(Copper)

= 23

= 36 = 1.98586

fian = 1.98787

fe = 5

metric mean = 1.96762

:iance = 0.0713494

indard deviation = 0.267113

indard error = 0.0556969

imum = 1.43508

:imum = 2.56495

ge = 1.12986

er quartile = 1.80829

er quartile = 2.17475

erquartile range = 0.366463

mess = -0.263077

1. skewness = -0.515077

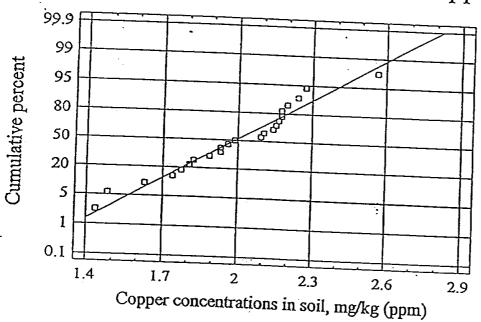
tosis = 0.18883

1. kurtosis = 0.184854

if. of variation = 13.4528

= 45.6679

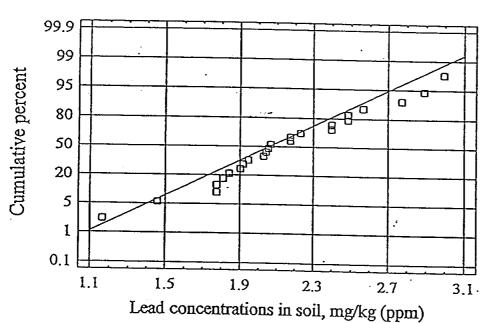
Lognormal Probability Plot for Copper



ummary Statistics for log(Lead)

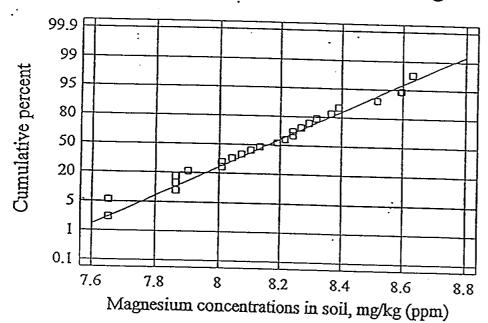
verage = 2.13936 edian = 2.06049 ode = eometric mean = 2.09509 ariance = 0.187882 tandard deviation = 0.433454 tandard error = 0.0884784 inimum = 1.16315 aximum - 2.99573 inge = 1.83258 wer quartile = 1.87133 per quartile = 2.4414 iterquartile range = 0.570072 :ewness = 0.0350174 ind. skewness = 0.0700348 rtosis = 0.200156 ind. kurtosis = 0.200156 eff. of variation = 20.261 m = 51.3446

Lognormal Probability Plot for Lead



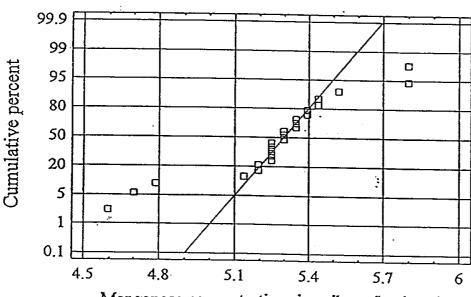
Statistics for log(Magnesium) erage = 0.14232 dian = 8.16011 ometric mean = 8.13815 riance = 0.0706013 andard deviation = 0.265709 andard error = 0.05423761imum = 7.64969 <imum = 8.63052</pre> ige = 0.980829 ver quartile = 7.95369 er quartile = 8.3064 erquartile range = 0.352709 wness = -0.0600481 d. skewness = -0.120096 tosis = -0.414246d. kurtosis = -0.414246 ff. of variation = 3.26331 = 195.416

Lognormal Probability Plot for Magnesium



mmary Statistics for log(Manganese) unt = 24 ecage = 5.2733 dian = 5.29832 ometric mean = 5.2661 riance = 0.0771874 andard deviation = 0.277826 andard error = 0.056711 nimum - 4.59512 ximum - 5.79909 nge = 1.20397 rer quartile = 5.21999 per quartile = 5.39363 erquartile range = 0.173637 wness = -0.660387 id. skewness = -1.32077 :tosis = 1.62566 id. kurtosis = 1.62566 :ff. of variation = 5.26854 1 = 126.559

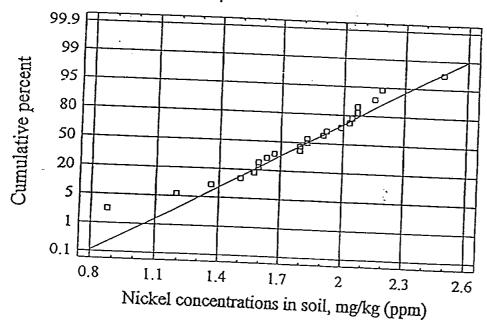
Lognormal Probability Plot for Manganese



Manganese concentrations in soil, mg/kg (ppm)

ummary Statistics for log(Nickel) **4** 23 age = 1.78451edian = 1.82455 eometric mean = 1.74596 riance = 0.1246 ;andard deviation = 0.352987 andard error = 0.0736029 nimum = 0.875469 ximum = 2.48491 nge = 1.60944 wer quartile = 1.58924 per quartile = 2.04122 terquartile range = 0.451985 ewness = -0.609856 nd. skewness = -1.19403 rtosis = 0.992502 nd. kurtosis = 0.971605 of variation = 19.7806 n = 41.0438

Lognormal Probability Plot for Nickel

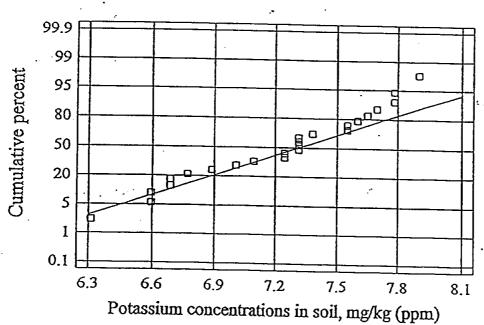


immary Statistics for log(Potassium) ount = 24 /erage = 7.21062 adian = 7.31322 ide = 7.31322 ometric mean = 7.20542 ciance = 0.195599 andard deviation = 0.442265 andard error = 0.0902771 nimum = 6.30992ximum = 7.90101nge = 1.59109 wer quartile = 6.82802 per quartile = 7.57526 terquartile range = 0.747233 ewness = -0.373735 nd. skewness = -0.74747ctosis = -0.83864nd. kurtosis = -0.83864

aff. of variation = 6.12673

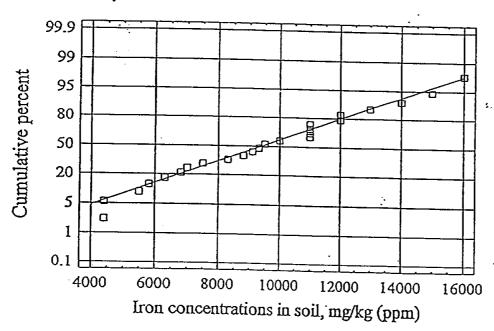
a = 173.247

Lognormal Probability Plot for Potassium



Hummary Statistics for Iron ≃ 24 age = 9529.17 edian = 9400.0 ode = 11000.0 cometric mean = 8977.5 ariance = 1.0363E7 tandard deviation = 3219.17 candard error = 657.109 inimum = 4400.0aximum = 16000.0 inge = 11600.0 wer quartile = 6900.0 per quartile = 11500.0 iterquartile range = 4600.0 :ewness = 0.20025 nd. skewness = 0.400499 rtosis = -0.620589 nd. kurtosis = -0.620589 eff. of variation = 33.7822 m = 228700.0

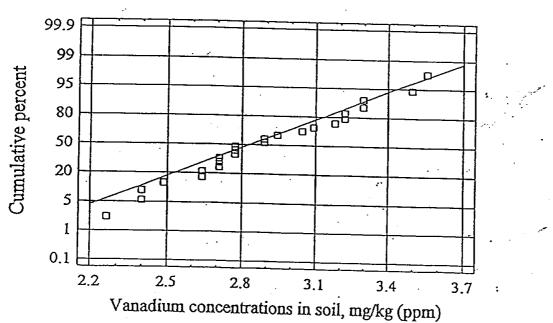
Normal Probability Plot for Iron



ummary Statistics for log(Vanadium) ount = 24 vecage = 2.89094 adian = 2.83148 ode 🖛 sometric mean = 2.87064 3riance = 0.122444 candard deviation = 0.34992 :andard error = 0.0714271 .nimum = 2.26176 x1mum = 3.55535 inga = 1.29358 wer quartile = 2.67355 per quartile = 3.19846 terquartile range = 0.524911 ewness = 0.158415 nd. skewness = 0.316831 rtosis = -0.688491 nd. kurtosis = -0.688491 eff. of variation = 12.104

m = 69.3826

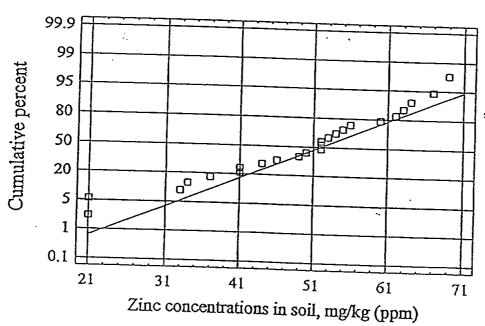
Lognormal Probability Plot for Vanadium



Summary Statistics for Zinc - 24 lan = 52.0 Yode = 52.0 Jeometric mean = 46.9434 /ariance = 171.478 standard deviation = 13.095 tandard error = 2.673 (Inimum = 21.0 (aximum = 69.0);ange = 48.0 ower quartile = 41.0 pper quartile = 58.0 nterquartile range = 17.0 kewness = -0.633044 tnd. skewness = -1.26609 urtosis = -0.0224531 end. kurtosis = -0.0224531 peff. of variation = 26.7244

ım - 1176.0

Normal Probability Plot for Zinc



Results	
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	Betyllium	S	0.3	QΝ	g	4.0	4.0	6.3	2	0.5	0.5	0.3	0.3	0.3	0.3	ΩN	-	0.4	0.3	0.3	0.3	╀-	╂-	- -		4.0
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	Sample Identifier	Bkg-01-A	Bkg-01-B	BKg-02-A	Bkg-02-B	Bkg-03-A	BKg-03-B	BKg-04-A	0-t-0-6va	A-co-by-d	B-cO-gya	A-00-Bya	BKG-00-B	BKG-07-A	BKG-07-B	BKG-08-A	H-80-6XH	BKG-09-A	akg-09-B	Bkg-10-A	Bkg-10-B	Bkg-11-A	Bkg-11-B	╀	╀	4

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Concentrations in mg/kg Activities in pCi/g Sample Identifier XX-XX-A - surface soil samples Sample Identifier XX-XX-B - subsurface soil samples Local Banground Soil Results

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	muilledT	S	2	9	2	2	2	2	QΝ	g	2	9	2	2	9	QN	Q.	2	L_	Q.	2	<u> </u>	2	2	QN	
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-	Selenium	QN	ND	QN	S	QN	Q	2	g	QN	QN	QΝ	QN	QN	ΔN	S S	9	ND	ND	QN	QN	QN	Q.	ND	ND	ĺ
L	muisseto9	1500	2000	730	1600	1500	1200	1900	1400	2700	1400	1500	800	870	800		980	$\overline{}$	550	2400		2100	2400	1500	1900	
L	Иіске	4	ω	7	က	7	െ	12	2	6	8	13	9	ည	ည	က	2	8	ည	9	\neg	7	8	9	8	
	Sample Identifier	Bkg-01-A	Bkg-01-B	Bkg-02-A	Bkg-02-B	Bkg-03-A	Bkg-03-B	Bkg-04-A	Bkg-04-B	Bkg-05-A	Bkg-05-B	Bkg-06-A	Bkg-06-B	Bkg-07-A	Bkg-07-B	Bkg-08-A	Bkg-08-B	Bkg-09-A	Bkg-09-B	Bkg-10-A	Bkg-10-B	Bkg-11-A	Bkg-11-B	Bkg-12-A	Bkg-12-B	
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Concentrations in mg/kg
Activities in pCi/g
Sample Identifier XX-XX-A - surface soil şamples
Sample Identifier XX-XX-B - subsurface soil samples

Normal Parameters for Tijeras Arroyo Local Metal Background Data Aluminum Chromium Antimony Manganese Cadmium N Arsenic Vanadium Statistical Copper Parameter Lead median 4300 8.5 140 2 6 geometric mean 4.2 7.3 9400 4579.9 7.9 200 8.6 6.2 17 3 144 2 52 5 3.7 maximum 7.3 8977.5 8.5 10000 195 16 210 3 10 6.6 13 16000 minimum 20 2200 330 4.4 12 2 35 95 69 1 2 arithmetic average 4970.8 0.1 4.2 4400 3.2 99 9 2.4 9.6 3 149 2 standard deviation 5.5 4.2 7.5 9529.2 2095.4 9.3 202 6.3 2 19 40.5 1 2.3 normal tolerance 1.3 2 3219.2 4.2 2.309 53.6 2.1 2 6.9 2.33 13 2 2.3 2.3 2.3 UTL 2.309 2.3 4927.4 2.31 2.3 244 2.3 11 7.3 12 16962 19 326

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Parameter	=	15	Arse	Barlum	Cadr	Chro	용	ddo	o L	äd	Mangane	Nickel	nadium	6	
arithmetic average	8.4294	2.2	1	4.97		_	O	U.		Le Le	ΙĔ	💆	a l	Zinc	l
standard deviation	0.4126	0.3	1	0.27	0	1.6	1.3		9.1025	2.1	5.27	1.8	2.9	3.8	
normal tolerance	2.309	2.3	2	2.33	2	0.5 2.3	0.8		0.3631	0.4	0.28	0.4	0.3	0.3	
UTL	9:3821	2.9	2	5.6	1	2.7	2.3	2.3	2.309	2.3	2.31	2.3	2.3	2.3	
e	11874	19	10	271	4	14	21	2.6	9.941	3.1	5.91	2.6	3.7	4.6	
I=						<u>-: T</u>]	41	14	20764	23	370	14	40	98	

Insufficient data for mercury, selenium, silver, and thallium to calculate statistics All concentrations in mg/kg

Summary of Background Concentrations for Radionuclides in Soil

	0		(August	2.7	9.0		, 60	edelection fimit	(<0.0586)	edetection limit	(<0.0418)	6.8	•		\		0.568		20.	1.	0 755					0.	0.168	?
95º Upoer					_	<u> </u> -		- ¢	×	v v v	-		95		+					-	6	1					°	_
958		· <u>@</u>				Ľ		_				'	1.0795	06.0	36.36	3	1	1.94	•	'	'	1 360	10.21			! . 	١	
	Median	(6CVg)	-		9.0	1	. 0.2495	detection limit	(<0.0686)	detection limit (<0.0418)	2835	200	0.5	0.56	164		0,635	0.590	0.630	ı	0.2883	0.810	23.		ı	6.0	0.1235	
	Geometric Mean	(pC/g)	1.1055	1	0.648		0.200	Coelection limit	(9990)	<0818010 (40.0418)	2,26838	089070	7.000A	0.549	15.889	0.6747		0.713	0.695	t	0.2528	0.7971	0.7796		-	0.897	0.1198	
		c	17	33	3,	1	504	7/1	5,6	·	46	333		240	718	72	12	5	2	0	45	136	35	-	,	4	50	
	Ranga	(0,04)	0.414-2.7	0.27-1.4		1 200	cratection limit	(<0.0688)	- Adetection limit	(<0.041B)	0,3-12,0	0.1-1.4	67.000	61,1 -82, 0	0.192-31.0	0.43-0.97	0.5-2.09	30, 37, 0	50,1-64,0	•	0.0321.85	0.23-1.20	0.324~3.0		0,000	0.1-0.0	0.05~0.18	
	Distribution Type		Nonparametric	Nonparametric	1	Nongarametric	Unknown		Unknown		Nonparametric	Lognormal	Loonormal	TO THE PERSON NAMED IN	Normal	Nonparametric	Lognormal	Nonparametric	- Interval	Monographic	notiparametric	Lognormai	Lognormal	Unknown	Nonogramatric	Monagement	included and all lo	
	Number of Rejected Samples	200		5.	26		ı		74	800	. 767	90	61			0	314	0	0	G		>	330	0	0	7.5		
	Number of Detects	12	224	3	561	ı	ı		-	07	333	253	241	720		3	53	24	0	45	136	3	35	0	4	21	206	
Original	Samples	324	340		802	•	ı		32.1	338	323		249	722	76		208	24	0	54	136	385			7	\$5	223	
	Analyte	Bismuth-212	Bismuth-214	Casim, 523	(Surface)	(Subsurface)	/222	Coball go		Lead-210*	Lead-212*	1 pad.214*	* 1 7.000	Polassium-40	Radium-224	Radium 22e	237	Hadium-228	Radon	Strontium-90	Thorium-232	Thorium-234	Trition		Uranium-234	Uranium-235	Uranium-238	

'Sample size. 'Sample size. 'Constituents of concern are of unknown distribution type because data are either below the limit of detection, unusable, or nonexistent.

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